

# Rocky Flats Environmental Technology Site

# PRE-DEMOLITION SURVEY REPORT (PDSR)

**Building 771 Locker Room Area (AC)** 

**REVISION 0** 

July 7, 2004

# CLASSIFICATION REVIEW NOT REQUIRED PER EXEMPTION NUMBER CEX-005-02



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## **REVISION 0**

July 7, 2004

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#### ABBREVIATIONS/ACRONYMS

ACM Asbestos Containing Material

Be Beryllium

CDPHE Colorado Department of Public Health and the Environment DCGL<sub>EMC</sub> Derived Concentration Guideline Level – elevated measurement

comparison

DCGL<sub>w</sub> Derived Concentration Guideline Level – Wilcoxon Rank Sum Test

D&D Decontamination and Decommissioning

DDCP Decontamination and Decommissioning Characterization Protocol

DOE U.S. Department of Energy DPP Decommissioning Program Plan

DQA Data quality assessment DQOs Data quality objectives

EPA U.S. Environmental Protection Agency
FDPM Facility Disposition Program Manual
HVAC Heating, ventilation, air conditioning
HSAR Historical Site Assessment Report
HEUN Highly Enriched Uranyl Nitrate
IHSS Individual Hazardous Substance Site
IWCP Integrated Work Control Package

K-H Kaiser-Hill

LBP Lead-based paint LLW Low-level waste

MARSSIM Multi-Agency Radiation Survey and Site Investigation Manual

MDA Minimum detectable activity
MDC Minimum detectable concentration
NORM Naturally occurring radioactive material

NRA Non-Rad-Added Verification

OSHA Occupational Safety and Health Administration

PARCC Precision, accuracy, representativeness, comparability and completeness

PCBs Polychlorinated Biphenyls
PDS Pre-demolition survey
PDSR Pre-demolition survey report

QC Quality Control

RCRA Resource Conservation and Recovery Act

RFCA Rocky Flats Cleanup Agreement

RFETS Rocky Flats Environmental Technology Site

RFFO Rocky Flats Field Office

RLC Reconnaissance Level Characterization

RLCR Reconnaissance Level Characterization Report

RSA Removable Surface Activity

RSOP RFCA Standard Operating Protocol RSP Radiological Safety Practices

SVOCs Semi-volatile organic compounds

TCLP Toxicity Characteristic Leaching Procedure

TSA Total surface activity

VOCs

Volatile organic compounds Waste Stream and Residue Identification and Characterization **WSRIC** 

## **EXECUTIVE SUMMARY**

A Pre-Demolition Survey was performed to enable compliant disposition and waste management of the Building 771 Locker Room area (referred to herein as Area AC). Because this area will be demolished, the characterization was performed in accordance with the Pre-Demolition Survey Plan (MAN-127-PDSP). Building surfaces characterized as part of this PDS include the interior surfaces of Area AC.

The PDS encompassed both chemical and radiological characterization. The characterization was built upon physical, chemical and radiological hazards identified in the facility-specific B771 and B774 Hazards Characterization Report for the 771 Closure Project.

Based upon the results of this PDSR, Area AC meets the unrestricted release limits specified in the site Pre-Demolition Survey Plan. This structure can be demolished and the waste managed as PCB Bulk Product waste or as sanitary waste, and the concrete can be used for backfill on-site per the RFCA RSOP for Recycling Concrete. To ensure that the facility remains free of contamination and PDS data remain valid, Level 2 isolation controls are established.

#### 1 INTRODUCTION

A Pre-Demolition Survey was performed to enable compliant disposition and waste management of the Building 771 Locker Room area (referred to herein as Area AC). Because this Type 3 building will be demolished, the characterization was performed in accordance with the Pre-Demolition Survey Plan (MAN-127-PDSP). The results of this survey shall demonstrate that Area AC meets the unrestricted release limits specified in the site Pre-Demolition Survey Plan. Building surfaces characterized as part of this PDS include interiors surfaces of Area AC.

As part of the Rocky Flats Environmental Technology Site (RFETS) Closure Project, numerous facilities will be removed. Among these is Area AC. This facility no longer supports the RFETS mission and will be removed to reduce Site infrastructure, risks and/or operating costs.

Before this Type 3 facility can be demolished, the Data Quality Objectives (DQOs) for a Pre-Demolition Survey (PDS) must be satisfied; this document presents the PDS results for Area AC. The PDS was conducted pursuant to the Decontamination and Decommissioning Characterization Protocol (MAN-077-DDCP) and the Pre-Demolition Survey Plan for D&D Facilities (MAN-127-PDSP). The PDS is built upon physical, chemical and radiological hazards identified in the facility-specific B771 and B774 Hazards Characterization Report for the 771 Closure Project, dated June 12, 2001, Revision 0.

#### 1.1 PURPOSE

The purpose of this report is to communicate and document the results of Area AC. A PDS is performed prior to building demolition to define the pre-demolition radiological and chemical conditions of a facility. The pre-demolition conditions are compared with the release limits for radiological and non-radiological contaminants. PDS results will enable project personnel to make final disposition decisions, develop related worker health and safety controls, and estimate waste volumes by waste types.

#### 1.2 SCOPE

This report presents the pre-demolition radiological and chemical conditions of the Area AC surfaces that will be free-released and disposed of as sanitary waste, recycle metal, or used as backfill per the requirements of the RFETS, RFCA RSOP for Recycling Concrete.

## 1.3 DATA QUALITY OBJECTIVES

The Data Quality Objectives (DQOs) used in designing this PDS meet the minimum requirements specified in Section 2.0 of the Pre-Demolition Survey Plan for D&D Facilities (MAN-127-PDSP). Refer to section 2.0 of MAN-127-PDSP for these DQOs.

#### 1.3.1 The Problem

The problem involves determining whether or not the survey unit is suitable for unrestricted release in accordance with this plan.

#### 1.3.2 The Decision

The decision is verification that objectives specified in the decommissioning decision document have been met (e.g., certain materials meet unrestricted release criteria for radiological and non-radiological constituents).

## 1.3.3 Inputs to the Decision

Inputs to the decision include the magnitude and location of data from preceding characterizations, including RLC and In-Process Characterization (IPC), PDS results, decision document action levels, and unrestricted release criteria.

#### 1.3.4 Decision Boundaries

The decision boundaries are the spatial confines of the facility, including rooms and sets of rooms, in two and three dimensions. Interior and exterior surfaces are included, including those below grade. Boundaries may be further defined in RFCA decision documents.

#### 1.3.5 Decision Rules

The following are decision rules to be used during PDS:

#### 1.3.5.1 Radionuclides

If all radiological survey and scan measurements (and sample measurements, where sample activity is translated to surface activity as described in Section 7.2.3 of the Pre-Demolition Survey Plan for D&D Facilities (MAN-127-PDSP)), are below the surface contamination guidelines specified in the Site PDSP, then the related areas and/or volume are considered not radiologically contaminated. The media sample result is calculated by converting volumetric activity (typically reported in pCi/g) to surface activity (dpm/100 cm²). The volumetric result (pCi/g) is multiplied by the weight of the sample (grams) and by 2.22 (conversion from pCi to dpm).

If any radiological survey or scan measurement exceeds the surface contamination guidelines provided in the Pre-Demolition Survey Plan for D&D Facilities (MAN-127-PDSP), the related survey unit must be evaluated per the statistical tests described in section 7.0, Data Analysis and Quality Assessment, of this plan. If any radiological sample measurement (or disposal unit volume) exceeds 100 nanocuries per gram of transuranic material, the related volume of material is considered transuranic (TRU) waste.

#### 1.3.5.2 Hazardous Waste

If decommissioning waste is mixed with or contains a listed hazardous waste, or if the waste exhibits a characteristic of a hazardous waste, then the waste is considered RCRA-regulated hazardous waste in accordance with 6 CCR 1007-3, Parts 261 and 268.

#### 1.3.5.3 Hazardous Substances

If material contains a listed hazardous substance above a decision document action level (e.g., RFCA) and/or the CERCLA reportable quantity (40 CFR 302.4), the material is subject to CERCLA regulation (i.e., remediation and/or notification requirements).

## 1.3.5.4 Beryllium

If surface concentrations of beryllium are equal to or greater than  $0.2 \,\mu\text{g}/100 \,\text{cm}^2$ , the material is considered beryllium contaminated per 10 CFR 850.

#### 1.3.5.5 PCBs

If material contains PCBs, in a non-liquid state, from the manufacturing process at concentrations ≥50 ppm, the material is considered PCB Bulk Product Waste and subject to the requirements of 40 CFR 761.

If PCB contamination from a past spill/release is suspected, or if a PCB spill is discovered that has not been cleaned up, the associated material is considered PCB Remediation Waste and subject to the requirements of 40 CFR 761. PCB remediation waste includes: materials disposed of prior to April 18, 1978, that are currently at concentrations ≥50 ppm PCBs, regardless of the concentration of the original spill; materials which are currently at any volume or concentration where the original source was ≥500 ppm PCBs beginning on April 18, 1978, or ≥50 ppm PCBs beginning on July 2, 1979; and materials which are currently at any concentration if the PCBs are spilled or released from a source not authorized for use under 40 CFR 761.

If a waste or item contains PCBs in regulated concentrations, the waste or item is classified as PCB-regulated material and subject to the requirements of 40 CFR 761.

#### 1.3.5.6 Asbestos

If any one sample of a sample set representing a homogeneous medium results in a positive detection (i.e., >1% by volume), then material is considered ACM (40 CFR 763 and 5 CCR 1001-10).

#### 1.3.6 Tolerable Limits on Decision Error

Acceptable false negative (a) errors for calculating the number of samples generally range from 1% to 10%. The default value specified by the Site PDSP is 5%, which was assumed for the survey design in this report.

## 1.3.7 Optimization of Plan Design

Statistically based radiological surveying and sampling will be conducted per the guidance in Appendix B of the RFETS Pre-Demolition Survey Plan for D&D Facilities (MAN-127-PDSP). Refer to Section 4.0 of the PDSP for direction of characterization of non-radiological, chemical constituents. For this report, the minimum number of

measurement locations is fifteen per survey unit, as calculated based on the guidance in MAN-127-PDSP. The DCGL<sub>W</sub> is 100 dpm/100 cm<sup>2</sup> for TSA and media measurements/samples, and 20 dpm/100 cm<sup>2</sup> for RSA measurements. The LBGR was adjusted to obtain a relative shift of two. The estimated standard deviation for each measurement type was calculated based on an assumed coefficient of variation of 30%.

The scan requirements for specific survey unit classifications are as follows:

Class 1: 100% of accessible surface Class 2: 10-100% floors/lower walls

2: 10-100% floors/lower walls 10-50% upper walls/ceilings

No Class 3 survey units are included in the scope of this report.

#### 2 HISTORICAL SITE ASSESSMENT

A facility-specific Hazards Characterization Report was conducted to understand the facility history and related hazards. The Building 771 Hazards Characterization was performed in June 2001 (Refer B771 and B774 Hazards Characterization Report for the 771 Closure Project, dated June 12, 2001, Revision 0). Based on the characterization results, no radiological contamination was identified in Area AC. The media sample results did not indicate radiological contamination in excess of the unrestricted release limits in or under the paint. However, Area AC is considered a Type 3 facility based on its proximity to Building 771.

The area included in the scope of this PDSR is referred to herein Area AC. This area was part of the original building 771 construction, and included the men's and women's locker rooms, janitor's closet, the portion of Corridor A just north of the Contamination Area airlock, and the northwest section of Corridor H. All non-load-bearing walls were removed from Area C during D&D activities.

Area AC consists of two Class 1 survey units (771034 and 771104), and one Class 2 survey unit (771110) based the contamination potential, per Section 3.0 of the PDSP.

The hazards characterization results and historical review (refer to Attachment G) were used to identify PDS data gaps and needs, and to develop radiological and chemical PDS characterization packages. Characterization documentation is located in the Building 771 Characterization Project files.

## 3 RADIOLOGICAL CHARACTERIZATION AND HAZARDS

Area AC was characterized for radiological hazards per the PDSP. Radiological characterization was performed to define the nature and extent of radioactive materials that may be present on the facility surfaces. Measurements were performed to evaluate the contaminants of concern (weapons-grade plutonium isotopes). Based upon a review of the characterization data, historical and process knowledge, in-process survey data, building walk-downs, and the Site Pre-Demolition Survey Plan (MAN-127-PDSP), a Radiological Characterization Plan was developed during the planning phase that

describes the minimum survey requirements (refer to survey packages 771034, 771104, and 771110). A Survey Unit Overview Map is presented in Attachment A. Based on hazard characterization data and historical and process knowledge, transuranic isotopes are the primary contaminants of concern in Buildings 771/774. Therefore, the PDS was performed to the transuranic PDS unrestricted release criteria. Individual radiological survey unit packages are maintained in the Building 771 Characterization Project files.

The Area AC survey unit packages were developed in accordance with Radiological Safety Practices (RSP) 16.01, Radiological Survey/Sampling Package Design, Preparation, Control, Implementation and Closure. Total surface activity (TSA) and removable surface activity (RSA) measurements were collected in accordance with RSP 16.02 Radiological Surveys of Surfaces and Structures. Radiological survey data were verified, validated and evaluated in accordance with RSP 16.04, Radiological Survey/Sample Data Analysis. Quality control measures were implemented relative to the survey process in accordance with RSP 16.05, Radiological Survey/Sample Quality Control.

Per the reference procedures, the required number of measurement locations is fifteen (15) per 100 square-meters of floor area for Class 1 survey units, and fifteen (15) per 1000 square-meters of floor area for Class 2 survey units. Scans were required on 100% of the structural surfaces for the Class 1 survey units; and 100% of lower wall and 10% of upper wall/ceiling surfaces for the Class 2 survey unit.

Radiological survey data, statistical analysis results, survey locations, and radiological scan maps are presented in Attachments B, C, and D, Radiological Data Summary and Survey Maps.

## Area AC Floors – (Survey Unit 771034)

The Area AC floor is a Class 1 survey unit. This area includes the floors of Rooms 120-128 (men's and women's locker rooms, janitor's closet, and the northwest section of Corridor H). This survey unit was originally clasified as Class 2, based on process history. However, contamination in excess of the unrestricted release limits was detected during the initial final survey effort. Hydrolazing water that leaked from the process areas of Building 771 is the suspected source. Because of the identification of areas of elevated activity, the classification was changed to Class 1. A total of 123 random TSA and RSA measurements were collected. Surface scans of 840 m<sup>2</sup> (100% of floor surfaces) were performed. All paint was removed from the floors, therefore no media samples were collected for this survey unit.

All scans and surveys in survey unit 771034 were less than the applicable PDS transuranic DCGL values. Radiological survey data, statistical analysis results, survey locations, and radiological scan maps for survey unit 771034 are presented in Attachment B, Survey Unit 771034 Radiological Data Summary and Survey Map.

## Area AC Walls/Ceiling-(Survey Unit 771110)

The walls and ceilings of Area AC are classified as a Class 2 survey unit. This area includes the walls/ceiling of Rooms 120-128 (men's and women's locker rooms, janitor's closet, and the northwest section of Corridor H). The classification was based on the low potential for contamination based on process history and characterization data. A total of 15 random TSA and RSA measurements and 15 media samples were collected. Surface scans of 229 m<sup>2</sup> of lower walls (100% of lower wall surfaces), 48 m<sup>2</sup> of upper walls (14% of upper walls surfaces), and 164 m<sup>2</sup> of the ceiling (14% of ceiling surfaces) were performed.

All scans, surveys, and media sample results in survey unit 771110 were less than the applicable PDS transuranic DCGL values. Radiological survey data, statistical analysis results, survey locations, and radiological scan maps for survey unit 771110 are presented in Attachment C, Survey Unit 771110 Radiological Data Summary and Survey Map.

## Area AC Northeast Corner – (Survey Unit 771104)

The northwest corner of Area AC is a Class 1 survey unit. This area includes the section of Corridor A that is located just outside of the former Process Area airlock. This survey unit was originally classified as Class 2, based on process history. However, contamination in excess of the unrestricted release limits was detected during the initial final survey effort. The source of contamination is suspected to be due to personnel, given that this was a high traffic area during building operations and due to the proximity of the area to the former Process Area. Because of the identification of areas of elevated activity, the classification was changed to Class 1. A total of 15 random TSA and RSA measurements and 15 media samples were collected. Surface scans of 180 m<sup>2</sup> (100% of total surface area) were performed.

All scans, surveys, and media sample results in survey unit 771104 were less than the applicable PDS transuranic DCGL values. Radiological survey data, statistical analysis results, survey locations, and radiological scan maps for survey unit 771104 are presented in Attachment B, Survey Unit 771104Radiological Data Summary and Survey Map.

#### 4 CHEMICAL CHARACTERIZATION AND HAZARDS

Based on a thorough review of historical and process knowledge, visual inspections, and personnel interviews, no additional chemical hazard sampling requirements were identified.

#### 4.1 Asbestos

Asbestos containing building material is not present in or on Area AC (previously removed).

## 4.2 Beryllium (Be)

Area AC is not and has never been a beryllium-controlled area. Per the Beryllium Sampling Decision Tree in the PDSP, seven (7) biased beryllium smear samples were collected in Area AC, in accordance with the PDSP and the *Beryllium Characterization Procedure*, PRO-536-BCPR, Revision 0, September 9, 1999.

All beryllium smear sample results were less than the investigative limit of  $0.1 \, \mu g/100 \text{cm}^2$ . PDS beryllium laboratory sample data and location maps are contained in Attachment E, Chemical Data Summaries and Sample Maps.

# 4.3 RCRA/CERCLA Constituents [including metals and volatile organic compounds (VOCs)]

Based upon the *B771 and B774 Hazards Characterization Report, 771 Closure Project*, Revision 0, dated June 12, 2001, personnel interviews, facility walk-downs, and historical process knowledge (WSRIC/WEMS), the Area AC did not contain hazardous waste storage units. A visual inspection of the building by 771/774 Industrial Hygiene personnel verified the absence of hazardous waste residuals and/or stains on the floor/concrete slab, walls, or ceiling. As a result of these observances, it has been determined that no sampling for RCRA/CERCLA constituents is required. The concrete generated from the demolition of the areas included in the scope of this report can be used for onsite recycling in accordance with the Concrete Recycling RSOP.

## 4.4 Polychlorinated Biphenyls (PCBs)

Based on historical knowledge, personnel interviews, and 771/774 Environmental Compliance Personnel walk-downs, Area AC never used/transferred free flowing/exposed PCB's. At one time the facility may have used PCB ballasts in its fluorescent light fixtures, however, all of these have been removed, and compliantly disposed of, resulting in no impact on demolition activities in this area.

Per the B771 and B774 Hazards Characterization Report for the 771 Closure Project, PCBs are present in some applied paints (i.e., on several walls and floors within the B771 and B774 Contamination Areas). Because additional paint sampling was not performed in Area AC, and because painted surfaces remain in the area (columns/walls/ceilings only), any painted debris generated during demolition that is not recycled on-site will be disposed of a PCB Bulk Product waste.

#### 5 PHYSICAL HAZARDS

Physical hazards associated with Area AC are common to standard industrial environments, and include hazards associated with utilities. There are no other unique hazards associated with the facility. The facility has been relatively well maintained and is in good physical condition, therefore, does not present hazards associated with building deterioration.

Physical hazards are controlled by the Site Occupational Safety and Industrial Hygiene Program, which is based on OSHA regulations, DOE orders, and standard industry practices.

## 6 DATA QUALITY ASSESSMENT

Data used in making management decisions for decommissioning of Area AC, and consequent waste management, is of adequate quality to support the decisions documented in this report. The data presented in this report (Attachments B, C, D, and E) were verified and validated relative to MAN-127-PDSP, Pre-Demolition Survey Plan for D&D Facilities, and original project DOOs.

In summary, the Verification and Validation (V&V) process corroborates that the following elements of the characterization process are adequate:

- ♦ the *numbe*r of samples and surveys;
- the *types* of samples and surveys;
- the sampling/survey process as implemented "in the field"; and
- the laboratory analytical process, relative to accuracy and precision considerations.

Details of the DQA are presented in Attachment F. The DQA Checklists are provided in the individual survey unit packages (located in the Building 771 Characterization Files).

The Minimum Detectable Activity (MDA) for each PDS instrument was determined a priori based on typical parameters (background, efficiency, and count time). A list of radiological field instrumentation and associated sensitivities is presented in Table 1.

Table 1
PDS Radiological Field Instrumentation and Minimum Detectable Activities

Model	Measurement Type	MDA (dpm/100 cm <sup>2</sup> )
NE Electra DP6	TSA	48
Eberline SAC-4	Removable (Smears)	10
NE Electra AP6	Scans	300

#### 7 DECOMMISSIONING WASTE TYPES

The demolition and disposal of Area AC will generate a variety of wastes. Concrete can be used as backfill onsite in accordance with the RFCA RSOP for Recycling Concrete.

#### 8 FACILITY CLASSIFICATION AND CONCLUSIONS

Based on the analysis of radiological, chemical and physical hazards, Area AC is classified as an RFCA Type 3 facility pursuant to the RFETS Decommissioning Program Plan (DPP; K-H, 1999). Based upon the results of this PDSR, Area AC meets the unrestricted release limits specified in the site Pre-Demolition Survey Plan and is ready

for demolition. The PDS for Area AC was performed in accordance with the DDCP and PDSP, all PDSP DQOs were met, and all data satisfied the PDSP DQA criteria.

A facility walkdown and historical review indicates that no RCRA/CERCLA constituents exist in Area AC (refer to Attachment G, Historical Review). Any painted debris generated during demolition that is not recycled on-site will be disposed of a PCB Bulk Product waste.

Radiological contamination in excess of the PDSP Table 7-1 limits was not detected in Area AC. The applicable limits are as follows:

Table 2
PDSP Table 7-1 Surface Contamination Limits

Radionuclides	Total Average (dpm/100 cm <sup>2</sup> ) (1) (DCGL <sub>W</sub> )	Total Maximum (dpm/100 cm <sup>2</sup> ) (DCGL <sub>EMC</sub> )	Removable (dpm/100 cm <sup>2</sup> ) (DCGL <sub>W</sub> )
Transuranics	100	300	20

(1) Measurements of average contamination should not be averaged over an area of more than 1 m<sup>2</sup>.

(2) The maximum contamination level applies to an area of not more than 100 cm<sup>2</sup>.

Based upon this PDSR, the Area AC can be demolished and the waste managed as sanitary and the concrete can be used for backfill on-site per the RFCA RSOP for Recycling Concrete.

To ensure that the facility remains free of contamination and that PDS data remain valid, Level 2 isolation controls have been established.

#### 9 REFERENCES

B771 and B774 Hazards Characterization Report for the 771 Closure Project, dated June 12, 2001, Revision 0.

DOE/RFFO, CDPHE, EPA, 1996. Rocky Flats Cleanup Agreement (RFCA), July 19, 1996.

DOE Order 5400.5, Radiation Protection of the Public and the Environment

DOE Order 414.1A, Quality Assurance

EPA, 1994. The Data Quality Objective Process, EPA QA/G-4.

K-H, 1999. Decommissioning Program Plan, June 21, 1999.

MAN-131-QAPM, Kaiser-Hill Team Quality Assurance Program, Rev. 1, November 1, 2001.

MAN-076-FDPM, Facility Disposition Program Manual, Rev. 3, January 1, 2002.

MAN-077-DDCP, Decontamination and Decommissioning Characterization Protocol, Rev. 4, July 15, 2002.

MAN-127-PDSP, Pre-Demolition Survey Plan for D&D Facilities, Rev. 1, July 15, 2002.

MARSSIM - Multi-Agency Radiation Survey and Site Investigation Manual (NUREG-1575, EPA 402-R-97-016).

PRO-475-RSP-16.01, Radiological Survey/Sampling Package Design, Preparation, Control, Implementation, and Closure, Rev. 1, May 22, 2001.

PRO-476-RSP-16.02, Pre-Demolition (Final Status) Radiological Surveys of Surfaces and Structures, Rev. 2, March 10, 2003.

PRO-477-RSP-16.03, Radiological Samples of Building Media, Rev. 1, May 22, 2001.

PRO-478-RSP-16.04, Radiological Survey/Sample Data Analysis for Final Status Survey, Rev. 1, May 22, 2001.

PRO-479-RSP-16.05, Radiological Survey/Sample Quality Control for Final Status Survey, Rev. 1, May 22, 2001.

PRO-563-ACPR, Asbestos Characterization Procedure, Revision 0, August 24, 1999.

PRO-536-BCPR, Beryllium Characterization Procedure, Revision 0, August 24, 1999.

RFETS, Environmental Waste Compliance Guidance #25, Management of Polychlorinated Biphenyls (PCBs) in Paint and Other Bulk Product Waste During Facility Disposition.

RFETS, Environmental Waste Compliance Guidance #27, Lead-Based Paint (LBP) and Lead-Based Paint Debris Disposal.

RFETS, RFCA RSOP for Recycling Concrete, September 28, 1999